

Session NG-I:
Theory and Methods
(Monday, October 23, p.m.)

NG-I/I1

J. Spanier

Research themes of broad transport applicability

(amc005)

NG-I/O1

T. Ueki, J.E. Hoogenboom, J.L. Kloosterman

Response Decomposition with Monte Carlo Correlated Coupling

(amc044)

NG-I/O2

T. Shuttleworth

The Geometrical Sensitivity Option in MCBEND

(amc121)

NG-I/O3

T.E. Booth

An alternative Monte Carlo method for the k-Eigenfunction?

(amc146)

NG-I/O4

J.E. Hoogenboom

Continuous Energy Adjoint Monte Carlo for Coupled Neutron-Photon Transport

(amc149)

NG-I/O5

A. Haghigat, J.C. Wagner

Application of A3MCNP to Radiation Shielding Problems

(amc211)

Session NG-II:
Code Status
(Monday, October 23, p.m.)

NG-II/I1

T. Mori, K. Okumura, Y. Nagaya

Status of JAERI's Monte Carlo Code MVP for Neutron and Photon Transport Problems

(amc027)

NG-II/I2

M.B. Emmett

Current Status of the Oak Ridge Monte Carlo Shielding Codes

(amc076)

NG-II/I3

N. Smith, T. Shuttleworth, M. Grimstone, L. Hutton, M. Armishaw, A. Bird, N. France, S. Connolly

The Current Status and Future Plans for the Monte Carlo Codes MONK and MCBEND

(amc119)

NG-II/I4

E.C. Selcow, G.W. McKinney

MCNP Capabilities at the Dawn of the 21st Century: Neutron-gamma Applications

(amc188)

NG-II/I5

L. Maiorov, E. Gomin, M.I. Gurevich

Status of MCU

(amc203)

NG-II/I6

J.C Nimal, T. Vergnaud

TRIPOLI-3 A Neutron/Photon Monte Carlo Transport Code

(amc238)

Session NG-III:
Physics & Data
(Tuesday, October 24, a.m.)

NG-III/I1

R. C. Little, S. C. Frankle, M. C. White, R. MacFarlane, C. J. Werner, J. M. Campbell

Modern Nuclear Data for Monte Carlo Codes
(amc235)

NG-III/O1

J. D. abrowska, K. Drozdowicz, U. Woznicka

The MCNP code in planning and interpretation of thermal neutron pulsed experiments
(amc054)

NG-III/O2

J. Tickner

Modelling Detector Responses to Neutrons using MCNP
(amc059)

NG-III/O3

J.P. Both, J.C Nimal, Y. Penelau

Water cross section sensitivity in the Naiade benchmark using TRIPOLI-4
(amc101)

NG-III/O4

P.A. Andresenko, M.R. Malkov

Simulation of thermal neutron transport processes directly from the Evaluated Nuclear Data Files
(amc116)

NG-III/O5

G. Hordósy, A. Keresztúri, Cs. Hegedus, P. Vértes

Importance of (γ n) reaction in the dynamic behaviour of Be reflected reactors
(amc171)

Session NG-IV:
Algorithms & Computational Science
(Tuesday, October 24, p.m.)

NG-IV/I1

K.W. Burn, G. Gualdrini, E. Nava

Variance Reduction with Multiple Responses

(amc092)

NG-IV/O1

T.J. Urbatsch, T.M. Evans, H. Lichtenstein

1-D Equilibrium Discrete Diffusion Monte Carlo

(amc043)

NG-IV/O2

C. Diop

Full Monte Carlo Particle Transport/Isotope Depletion Coupling

(amc118)

NG-IV/O3

J.L. Hutton

Use of a Hybrid Monte Carlo Technique for Power Shape Calculations

(amc122)

NG-IV/O4

O. Jacquet, X. Bay, L. Carraro

Evaluation of the Uncertainty in Eigenvalue Monte Carlo Calculations, Using Time Series Methodologies

(amc130)

NG-IV/O5

J. Miss, A. Nouri 1, O. Jacquet

Development and Comparison of Monte-Carlo Techniques for the Calculation of Loosely Coupled Systems

(amc131)

Session NG-V:
Experiments & Measurements
(Wednesday, October 25, a.m.)

NG-V/I1

B. Siebert

Sensitivity Analysis and Uncertainty Assessment in Applied Monte Carlo Particle Transport

(amc166)

NG-V/O1

B.T. Rearden, L.M. Petire, D.F. Hollenbach

Sensitivity and Uncertainty Analysis for Nuclear Criticality Safety Using KENO in the SCALE Code System

(amc231)

NG-V/O2

A. Fernandes, A. Vieira, I.C. Gonçalves, A. Ramalho

Monte-Carlo determination of the radiation field in the thermal column of the Portuguese Research Reactor

(amc050)

NG-V/O3

T. Aoyama, T. Sekine

Fission Rate Analysis in the Irradiation Test Subassemblies of Fast Reactor Using MCNP

(amc082)

NG-V/O4

T. Valentine

Benchmarking Monte Carlo Codes for Criticality Safety Using Subcritical Measurements

(amc018)

NG-V/O5

R.J. Tanner, D.T. Bartlett, L.G. Hager

Calculations of the Response of the NRPB Neutron Personal Dosemeter using MCNP and Experimental Verification of those Calculations

(amc123)

Session NG-VI:
Applications
(Wednesday, October 25, a.m.)

NG-VI/I1

T. Goorley

Review of Industrial and Medical Applications that Utilize Coupled Neutron-Photon Monte Carlo Transport

(amc057)

NG-VI/O1

I.F. Gonçalves, A.G. Ramalho, I.C. Gonçalves, J. Salgado

MCNP Calculations for the shielding design of a beam tube to be installed at the Portuguese Research Reactor

(amc041)

NG-VI/O2

U. Fischer

Monte Carlo Applications in Fusion Neutronics

(amc052)

NG-VI/O3

F. Rahnema, S. Mosher, M. Pitts, P. Akhtar, D. Serghiuta

Continuous-Energy Monte Carlo Simulation of a Simplified CANDU Core Sub-region

(amc056)

NG-VI/O4

M. Maucec, P.H.G.M. Hendriks, R.J. de Meijer

Monte Carlo Simulation of Natural Gamma Ray Spectrometry for Underwater Surfaces

(amc096)

NG-VI/O5

B. Verboomen, A.Beeckmans. de West-Meerbeeck, Th. Aoust, and Ch. De Raedt

Monte Carlo Modelling of the Belgian Materials Testing Reactor BR2

(amc142)

Session NG-VII:
Benchmarks
(Thursday, October 26, a.m.)

NG-VII/I1

S. Sitaraman, R-T. Chiang, K. Asano, K. Koyabu

Benchmark for a 3D Monte Carlo Boiling Water Reactor Fluence Computational Package - MF3D

(amc021)

NG-VII/O1

U. Fischer, I. Kodeli, C. Konno, R. L. Perel

Intercomparison of Monte Carlo and S_N Sensitivity Calculations for a 14 MeV Neutron Benchmark

(amc053)

NG-VII/O2

S.M. Zaritsky, N.I. Alexeyev, A.E. Glushkov, M.I. Gurevich, D.A. Shkarovsky

Comparison of calculations with benchmark experiments: validation of the MCU-4 code with ENDF/B-VI data library for tasks of neutron penetration through iron

(amc065)

NG-VII/O3

T. Goorley, S. Kiger, R. Zamenhof

Comparison of Snyder Head Phantom Models Used for Neutron Capture Therapy Benchmark Monte Carlo Dosimetry Calculations

(amc074)

NG-VII/O4

A. Polanski, V. Barashenkov, I. Puzynin, I. Rakhno, A. Sissakian

Monte Carlo modeling of fast sub-critical assembly with MOX fuel for research of accelerator driven systems

(amc168)

NG-VII/O5

Y.K. Lee

Evaluation of CRISTO II Storage Arrays Benchmark with TRIPOLI-4.2 Criticality Calculations

(amc180)

Session NG-VIII:
Graphics & Applications
(Thursday, October 26, a.m.)

NG-VIII/I1

R.A. Schwarz, L.L. Carter, V.E. Roetman, K.E. Hillesland
Current State of Monte Carlo Visualization Tools
(amc153)

NG-VIII/O1

T. Kugo
Conceptual Designing of a Reduced Moderation Pressurized Water Reactor by Use of MVP and MVP-BURN
(amc048)

NG-VIII/O2

M. Armishaw, A. Bird, H. Crofts, N. Smith
The Role of Graphical Supporting Tools for Monte Carlo Analysis
(amc120)

NG-VIII/O3

D.L. Bleuel, R.J. Donahue, B.A. Ludewigt, J. Vujic
Simulating Problems with Variable Sources via Post Processing of Individual Particle Tallies
(amc143)

NG-VIII/O4

Z. Karrien, C.C. Stoker, F. Reitsma
MCNP Modelling of HTGR Pebble-Type Fuel Elements
(amc150)

NG-VIII/O5

S.M. Bowman, J.E. Horwedel
KENO3D Visualization Tool for KENO V.a and KENO VI
(amc232)

NG-P (Poster Session)
Neutron-gamma contributed papers
(Wednesday, October 25, p.m.)

NG-P1

V.V. Korobeinikov

Universal approach for ADS blanket calculations by Monte Carlo method
(amc001)

NG-P2

T. Kato, T. Nakamura

Development of analytical method for bulk shielding calculation in a medium energy accelerator facility
(amc012)

NG-P3

H.J. Shim, C.S. Jang, Chang Hyo Kim

Monte Carlo Depletion Analysis of PWR Integral Fuel Burnable Absorbers by MCNAP
(amc025)

NG-P4

O. Sato, S. Mitake, H. Tsunoda

Application of Biasing Optimization Techniques to Monte Carlo Shielding Analysis of Transport Cask
(amc067)

NG-P5

L. Petrizzi, H. Iida, D. Valenza, P. Batistoni

Improvement and Benchmarking of the New Shutdown Dose Estimation Method by Monte Carlo Code
(amc097)

NG-P6

P.A. Androsenko, D.L. Joloudov, A.V. Kompaniyets, M.R. Malkov

Monte Carlo Simulation of Neutron and Photon Transport with Direct Utilization of Information from Evaluated Nuclear Data Files
(amc148)

NG-P7

W. Gudowski, J. Cetnar, J. Wallenius, K. Tucek

Simulation of nuclide transmutation with Monte-Carlo continuous energy burnup system (MCB)
(amc196)

NG-P8

R.-T. Chiang, S. Sitarman, K. Asano, K. Koyabu

Application of MCNP-based Software Package MF3D for BWR Radiation Damage Computations

(amc016)

NG-P9

D. Schlegel, S. Guldbakke

Why do we need TARGET ?

(amc037)

NG-P10

Y. Rugama, J.L. Munoz-Cobo, T.E. Valentine

Noise Method for Monitoring the Subcriticality in Accelerator Driven Systems

(amc073)

NG-P11

R.J. de Meijer, P.H.G.M. Hendriks, M. Maucec

Sediment-density Dependence of Natural Gamma Ray Spectra in a Borehole

Geometry

(amc155)

NG-P12

E. Bourhis, S. Hachem, J.-P. Pignol, N. Iborra Brassart, J. Herault, P. Chauvel, D. Paul, R. Sabattier, and N. Breteau

The use of MCNP4B for Monte Carlo simulation of the dosimetry of ^{10}B neutron capture enhancement of fast neutron

(amc173)

NG-P13

A. Rogov, Y. Pepyolyshev

Experimental and Monte Carlo study of the plastic scintillator performance for the current mode measurement at pulsed neutron sources

(amc177)

NG-P14

R. Cabezas, H. Yoriyaz, A. Santos, M. Yamaguchi

Monte Carlo Burnup Analysis of the Nuclear Research Reactor IEAR-1 Using MCNP-4B Code

(amc061)

NG-P15

H. Sawamura, K. Nishimura, S. Takahashi, J.S. Schweitzer

The Feasibility Study of Non-Radioactive Tracers for Monitoring Injected Water in Oil Reservoirs

(amc083)

NG-P16

S. Rollet

Neutronics and Shielding Analysis of Ignitor

(amc088)

NG-P17

J. Svarny

About the Application of MCNP4 Code in Nuclear Reactor Core Design Calculations

(amc094)

NG-P18

A. Vieira, I. C. Goncalves, A. Ramalho

Monte Carlo calculations for neutron and gamma radiation fields on a fast neutron irradiation device

(amc098)

NG-P19

M. Coeck, T. Aoust, F. Vermeersch, and H.A. Abderrahim

Shielding assessment of the MYRRHA accelerator driven system using the MCNP code

(amc139)

NG-P20

G. Hordósy, P. Vértes

Influence of gamma photons and thermal neutrons on reactor pressure vessel damage

(amc170)

NG-P21

S. Belousov, K. Ilieva

The MCNP Application vor VVER-1000 Shielding Calculation

(amc179)

NG-P22

J. Ghassoun, K. Ueki, A. Jehouani

The Fast and the Thermal Neutron Flux Distribution in TRIGA Reactor Shields by Monte Carlo Method

(amc195)

NG-P23

Victor Muratov and Aleksander Lopatkin

*Numerical tests for the problem of U-Pu fuel burnup in fuel rod and polycell models
using the MCNP code*

(amc241)